

OPTIMIZATION STUDIES OF A NEW DESIGN ARRAY DETECTORS USING MONTE CARLO SIMULATIONS

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Abstract

A development using different gamma spectrometers was constructed for enhancing detection and verification tools of nuclear materials and radioactive source. The array design consists of HPGe detector and three different NaI detectors; one of them represents the annual detector and the others represent the guard detector. The aim of the present work is studying designs in which the perfect position for the detectors with respect to each other as well as the sample by changing the guard detectors position along the x-axis. Where the Ge detector placed, to get the best position which gives the highest performance for the array. The Monte Carlo code (MCNP5) is used to get the optimum conditions to the design at which the systematic error is being as low as possible. The calculations require the knowledge and data of the internal dimensions of the used detectors and samples and the distances between them. The estimated results proved that the perfect position for the array when the guard detectors placed at zero angle from the x-axis where the Ge-detector placed.

Key Words: MCNP (Monte Carlo Code), SNM (standard Nuclear Material), HPGe, NaI.

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1. Introduction

The precision of gamma spectrometry is partly related to the efficiency calibration of the detector system. The detector efficiency is usually much less than 100% on account of the loss of gamma rays through interactions either within the sample itself, the detector or other inactive regions (e.g. detector packaging, electrical contacts, semiconductor deadlayer) [3]. Germanium detectors are often used as the primary detector due to their efficiency and energy resolution which is surrounded by three NaI detectors one of them represented as annular detector and the two others guard detectors. Further challenges to reach the optimum positions for such detectors with the sample

Monte Carlo simulations are a set of computational algorithms which rely on repeated random sampling and are utilized extensively in predicting the interaction of radiation with materials [3].

2. Monte Carlo Simulation

MCNP is a powerful tool used for calculating of energies and positions of different types of particles (for instance electrons, photons, neutrons and positrons) during their interaction with specific materials which have unique shape. The main problem of defining parameters for simulation in MCNP is to define sufficiently geometry of experimental setup to get as realistic model as possible. The input file of this code is created in which geometry, materials, particle source, type of results and numbers of histories are defined [9].

F8 tally is used in the input file which estimate the pulse height tally in each cell will be important and TRCL card is used as well which responsible for the transformation of the cell to different solid angles. The TRCL card makes it possible to describe just once the surfaces that bound several cells identical in size and shape but located at different places in the geometry.

TRn Coordinate Transformation Card

Form: TRn O1 O2 O3 B1 B2 B3 B4 B5 B6 B7 B8 B9 M

n = number of the transformation: 1 n 999.

*TRn means that the B_i are angles in degrees rather than being the cosines of the angles.

O1 O2 O3 = displacement vector of the transformation.

B1 to B9 = rotation matrix of the transformation.

$M = 1$ (the default) means that the displacement vector is the location of the origin of the auxiliary coordinate system, defined in the main system.

$= -1$ means that the displacement vector is the location of the origin of the main coordinate system, defined in the auxiliary system.

The B matrix specifies the relationship between the directions of the axes of the two coordinate systems. B_{ij} is the cosine of the angle (or the angle itself, in degrees in the range from 0 to 180, if the optional asterisk is used) between an axis of the main coordinate system (x, y, z) and an axis of the auxiliary coordinate system as follows:

Element $B1 B2 B3 B4 B5 B6 B7 B8 B9$

Axes $x, x', y, y', z, z', x', x', y', y', z, z', x, z', y, z', z, z'$ [10].

The model of the HPGe detector was based on the manufacturer specifications and design drawings. However, discrepancies between the real and simulated system can occur if technical information from the manual is omitted e.g. the deadlayer thickness. Limited accuracy in the actual set-up in the detector (e.g. errors in the distance of the crystal to the front window, misalignment of the crystal and its housing occurring during the manufacturing phase etc.) or from changes to the size of the detector components after cryogenic cooling can also be responsible for errors. Additionally, the deadlayer thickness may vary considerably over the detector surface and can also increase with time [3].

The annulus detector is positioned with alignment with the HPGe detector. The guard detectors placed at both sides of annular detectors, SNM and Ge detector as shown in the figures. The annulus NaI detector is dimensioned as 3x3 inch contained in a thin aluminum holder. One of The guard detectors dimensioned as 3x3 inch and other 2x2 inch. The detailed information are showed as in the figure (2.1)

HPGe detector were used as an initial guess in the simulations. Several parameters are important, including dead-layer thicknesses, to get closer agreement with the measurements [1].

Although employing the beta/gamma coincidence measurement technique significantly increases the sensitivity of the detection system and improves the measurement of ultra-low radioactivity in a high background field, there is still some probability for high-energy gamma-rays to be scattered through Compton scattering in the gamma detector. These interactions will add unwanted background for regions of interest at lower energies [6].

The main objective of this research is to integrate an ideal design array mechanism to further advance the detection and measurement by reducing the Compton background in the regions of interest of gamma-ray spectra.

This paper described a new method for gamma ray analysis by using radioactive source ^{137}Cs at energy line 661.7 KeV and SNM with enrichment 2.95% at energy line 185.7 KeV. The guard detectors are placed where the Al-caps are front of the Ge detector on the x-axis and then the angle between the guard detectors and the center of Ge detector will be increased gradually ($0^\circ, 10^\circ, 20^\circ, \dots, 90^\circ$). Each time, the net area is recorded as shown in the figure (4.1).

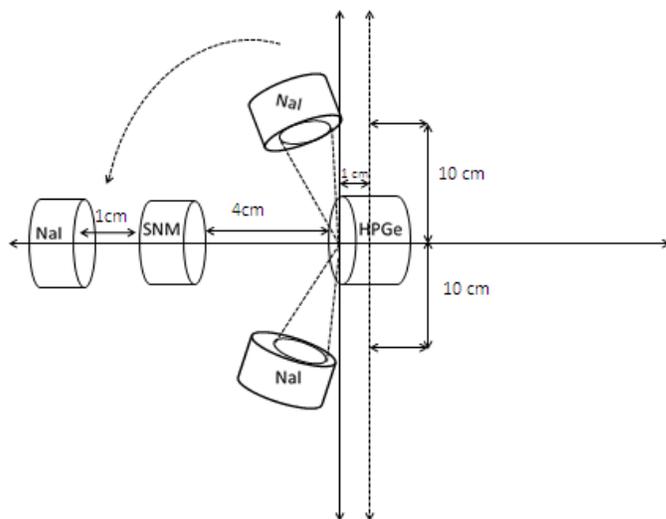


Fig.(4.1) the changes of guard detectors positions in the array

4. Results:

The detailed dimensions of the used detectors and the sample, as well as, the distances between the detectors and each other are used in the input files of MCNP code. The simulated array is shown as in the figure (4.2)

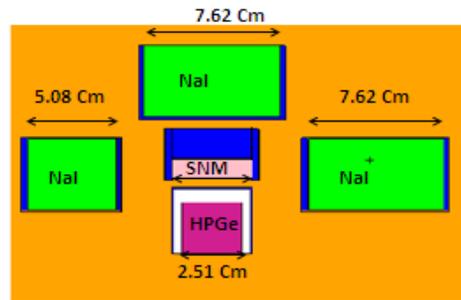


Fig.(4.2) the simulated array by MCNP5 where the guard detectors at angle 90° with x-axis Because there are a great number of assumptions and approximations made to model array detector and attempts to reach perfect position of guard detectors in the array. Many different predictions are executed MCNP transport model and comparing the results with each other for one sample. The results give highest count at the angle 0 as shown in the figures (4.3) for radioactive source and (4.4) for SNM.

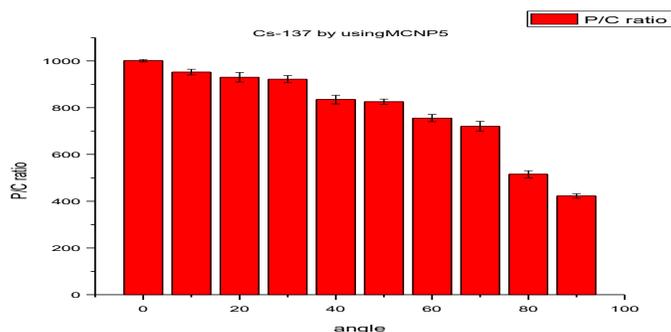


Fig.(4.3) the simulated results by MCNP5 by using radioactive source Cs-137

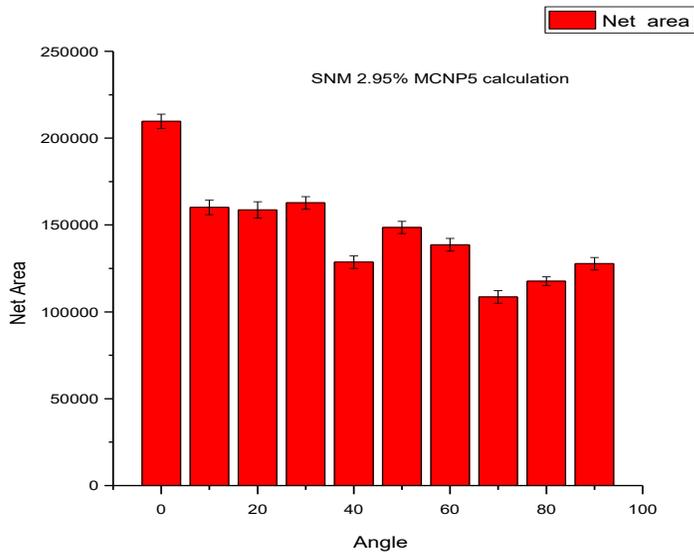


Fig.(4.4) the simulated results by MCNP5 by using SNM

5. Conclusion

In this paper, It was determined the best array parameters optimization using MCNP transport calculations. The height optimization demonstrates a maximum net peak area was obtained by the MCNP simulations. Tally F8 (detector tally) was used to calculate detector response in the active area. The results proved that the perfect position for the array is when the guard detectors (NaI) placed at zero angle from the x-axis, where the HpGe-detector is placed.

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